

Recent Activities & Summary of Study in Japan on Severe Accident Monitoring for Nuclear Power Plants

October 14-17th, 2013

IEEE-SC2 at Shanghai, China

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TOSHIBA CORPORATION
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“SA-計装(Kei-Sou)”

Stands for

“Severe Accident -
Instrumentations & Monitoring Systems”

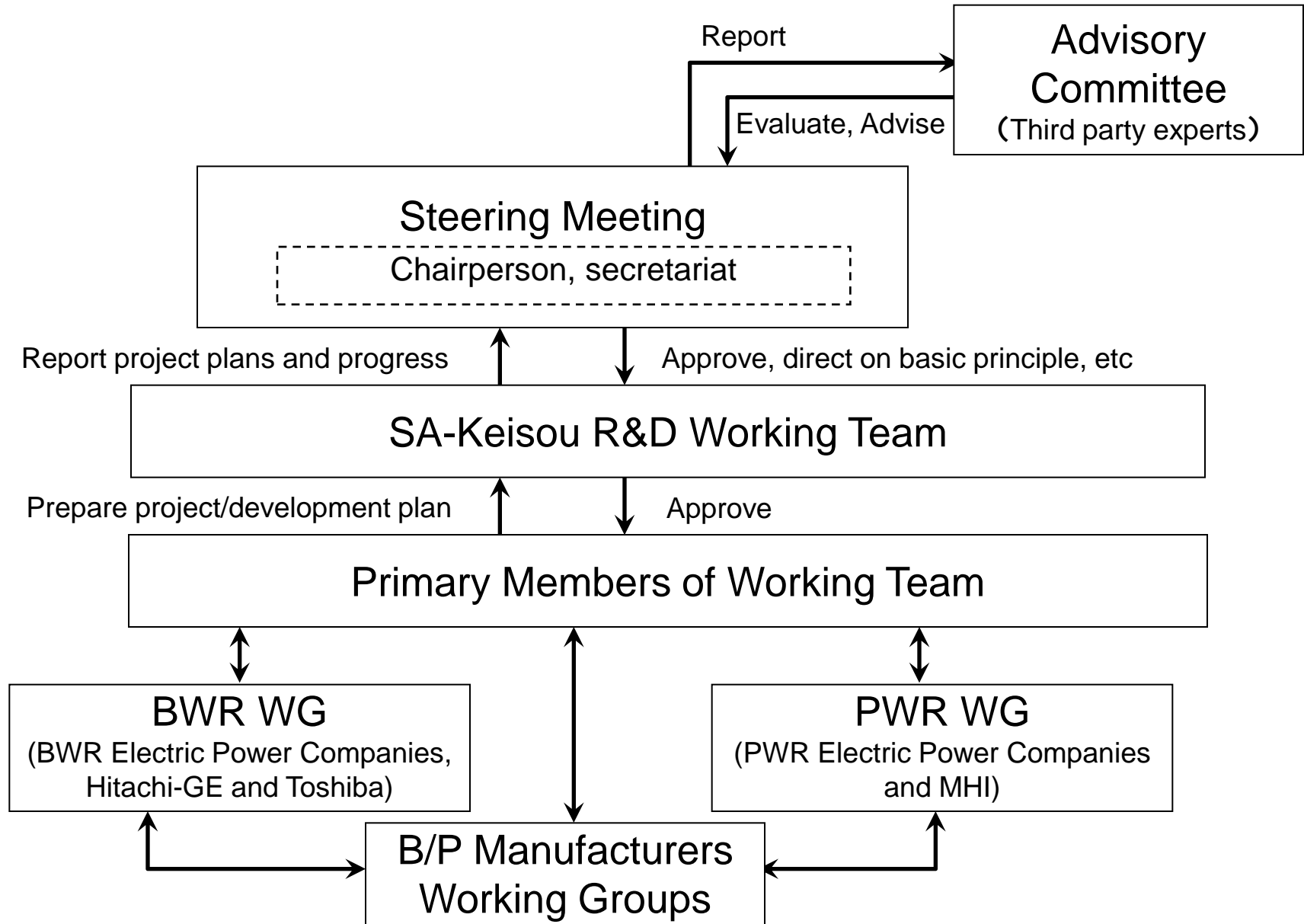
1. Summary of Study of Severe Accident Monitoring & Instrumentation System
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 - 1.3 SA Classification
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 - 1.5 SA Variables & SA Classification
 - 1.6 Design Requirements of Industrial Proposal
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Appendix: The latest version of SA Variables & SA Classification

1. Summary of Study of Severe Accident Instrumentations & Monitoring System

- Through this research & development program, the specific requirements for the SA-Keisou (Severe Accident instrumentations & monitoring systems) will be prepared.
- If a severe accident happens, the SA-Keisou could contribute to suppressing the accident by measuring the important parameters (e.g., water level, pressure, etc.).
- The program also enhance technical level in the nuclear industry in Japan and promote safety and utilization of the nuclear power plants.

1.2 Promoting Organization



1.3 SA Classification(1): Objective and Approach

● Objective of SA Classification

- Definition of typical SA condition corresponding to event progress after core damage.
- Determination of environment conditions and classification of measurement variables corresponding to SA conditions.

● Approach of determination of environmental conditions

- Selection of typical event trees indicating each SA condition.
- Determination of environmental conditions based on analysis using event trees.

● Approach of classification of measurement variables

- Determination of measurement variables corresponding to instrument purposes of SA conditions.
- Instrument purposes are to;
 - confirm plant conditions;
 - confirm initiation and success of operations for mitigating event.

1.3 SA Classification(2): Definition

● Classification of SA Conditions due to Event Progress

Plant conditions after the core damage are classified as
SA1, **SA2**, **SA3a**, and **SA3b**.

SA1: Core is damaged, but core fuel is inside the Reactor Vessel (RV).
(Accident Management RCS Pressure Reduction is Achieved.)

SA2: RV Failure, and core fuel is outside the RV
(AM Water Injection into the Containment Vessel and Containment Natural
Convection Cooling are Achieved.)

SA3a: Containment Failure ([Water Feed Injection to Inside of CV](#) is Success)

SA3b: Containment Failure ([Water Feed Injection to Inside of CV](#) is Failure)

Note) PWR(BWR): RV(RPV),CV(PCV)

● Environmental Conditions

★ Core Fuel, RV(RPV), CV(PCV) Conditions and Water Feed Injection vs. SA Classification

Condition	SA1	SA2	SA3a	SA3b
Fuel/Location	Meltdown /RV	Debris /RV or CV	Debris /RV or CV	Debris /RV or CV
Core	Damaged	Damaged	Damaged	Damaged
RV	Sound	Damaged	Damaged	Damaged
CV	Sound	Sound	Damaged	Damaged
<u>Water Feed Injection</u>	Success	Success	Success	Failure

Note) PWR(BWR): RV(RPV),CV(PCV)


1.3 SA Classification(4): Comparison with Level of Defense in Depth ¹⁰

SA classification [Ⓢ]		DBA/BDBA [Ⓢ]		SA1 [Ⓢ]	SA2 [Ⓢ]	SA3a [Ⓢ]	SA3b [Ⓢ]
Fuel condition [Ⓢ] /Fuel position [Ⓢ]		Within RPV [Ⓢ]		Meltdown [Ⓢ] /Within RPV [Ⓢ]	Debris [Ⓢ] /RPV or PCV [Ⓢ]	Debris [Ⓢ] /RPV or PCV	Debris [Ⓢ] /RPV or PCV [Ⓢ]
Core condition [Ⓢ]		Sound [Ⓢ]		Damaged [Ⓢ]	Damaged [Ⓢ]	Damaged [Ⓢ]	Damaged [Ⓢ]
RPV condition [Ⓢ]		Sound [Ⓢ]		Sound [Ⓢ]	Damaged [Ⓢ]	Damaged [Ⓢ]	Damaged [Ⓢ]
PCV condition [Ⓢ]		Sound [Ⓢ]		Sound [Ⓢ]	Sound [Ⓢ]	Damaged [Ⓢ]	Damaged [Ⓢ]
Water Feed Injection [Ⓢ]		Success [Ⓢ]		Success [Ⓢ]	Success [Ⓢ]	Success [Ⓢ]	Failure [Ⓢ]
IAEA level of [Ⓢ] defense in depth [Ⓢ]		Level 3 [Ⓢ]		Level 4 [Ⓢ]		Level 5 [Ⓢ]	
WENRA level of [Ⓢ] defense in depth [Ⓢ]		Level 3a [Ⓢ]		Level 3b [Ⓢ]		Level 4 [Ⓢ]	
Defense in depth layers [Ⓢ]		3 – 1 [Ⓢ]		3 – 2 [Ⓢ]		4 – 1 [Ⓢ]	
Plant condition [Ⓢ]		DBA and maintain safety function (assume single failure criterion). [Ⓢ]		Loss of safety function based on design basis (multiplex failure). [Ⓢ]		Significant core damage. [Ⓢ]	
Purpose of level of defense in depth [Ⓢ]		Prevention of significant core damage. [Ⓢ]		Prevention of massive emission. [Ⓢ] Prevention of containment vessel damage. [Ⓢ]		Massive emission. [Ⓢ] Containment vessel damage. [Ⓢ]	
Safety measures [Ⓢ]		ECCS, etc.. [Ⓢ]		Preventive measure of significant core damage (SA measure I). [Ⓢ]		Emission control and spreading mitigation. [Ⓢ]	
				Prevention of human damage. [Ⓢ] Restoration of environment. [Ⓢ]		Prevention of massive emission. [Ⓢ] Prevention of containment vessel damage. [Ⓢ]	
				Preventive measure of containment vessel damage (SA measure II). [Ⓢ]		Measure of emission control and spreading mitigation. [Ⓢ]	
						Disaster prevention. [Ⓢ]	

↑ Boundary between the third layer and the fourth layer

1.3 SA Classification(5):Relationship with Event Tree for BWR

Event Tree (Actions and Failures)

Large Scale Earthquake+Tsunami	RCIC 8Hr + Depressurization	External Water Feeding after 8Hr	External Water Feeding after 24Hr	SA Classification
<p>Representative Scenario </p>	SBO RCIC+Depressurization	RPV Water Feed+D/W Spray (Small LOCA)	D/W Spray	SA1
		RPV Water Feed (Small LOCA)	D/W Spray Failed	SA1
		Water Feed Failed	D/W Spray Failed	SA1
	TQUV Depressurization	Water Feed Failed	D/W Spray	SA3a
			D/W Spray Failed	SA3b
			RPV Water Feed+D/W Spray	SA2
		RPV Water Feed	D/W Spray	SA2
			D/W Spray Failed	SA2
			D/W Spray	SA3a
	Water Feed Failed	D/W Spray Failed	SA3b	

1.3 SA Classification(6): Relationship with Event Tree for PWR

Event Tree (Actions and Failures)

Large Scale Earthquake + Tsunami	Turbine Motored Auxiliary Water Feed	External Water Feeding after 8Hr (Auxiliary Water Feed)	External Water Feeding after 8Hr	External Water Feeding after 24Hr	Containment Atmosphere	Containment Conditions	SA Classification
Auxiliary Water Feed Achieved					(No Core Damage)	-	-
Achieved					Feed and Breed + CV Spray	(No Core Damage)	-
Failed ✖					CV Water Feed	Saturated	-
Failed ✖					CV Spray Failed	Saturated	Containment Failure
Failed ✖					Water Feed Failed	Overheated	Containment Failure
Failed ✖					CV Spray	Saturated	-
Failed ✖					CV Water Feed	Saturated	-
Failed ✖					CV Spray Failed	Saturated	Containment Failure
Failed ✖					Water Feed Failed	Overheated	Containment Failure

Representative Scenario (indicated by a red arrow pointing to the path: Failed ✖ → CV Water Feed → CV Spray Failed → Water Feed Failed)

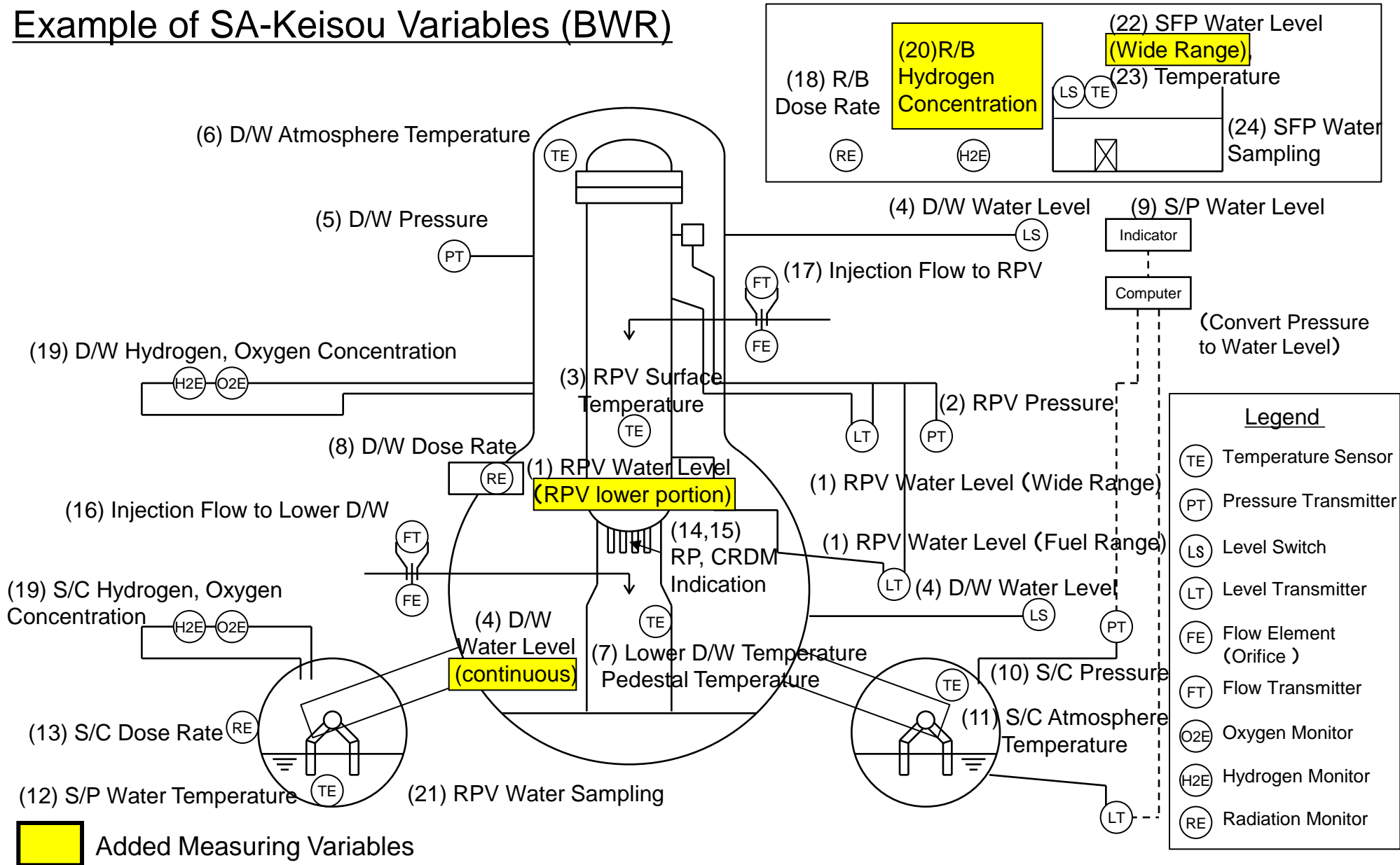
1.4 Selection of Measuring Variables(1) for BWR

● Selected Variables for BWR (Added Variables in red)

No.	Variables	No.	Variables
1	RPV Water Level (RPV lower portion)	13	S/C Dose Rate
2	RPV Pressure	14	Rod Position Indication Value
3	RPV Surface Temperature	15	CRDM Indication Value
4	D/W Water Level (Continuous)	16	Injection Flow to Lower D/W
5	D/W Pressure	17	Injection Flow to RPV
6	D/W Atmosphere Temperature	18	R/B Area Radiation Monitor
7	Pedestal Atmosphere Temperature	19	D/W, S/C Hydrogen, Oxygen Concentration
8	D/W Dose Rate	20	R/B Hydrogen Concentration
9	S/P Water Level	21	RPV Water Sampling Analysis <still under debate >
10	S/C Pressure	22	SFP Water Level (Wide Range)
11	S/C Atmosphere Temperature	23	SFP Water Temperature
12	S/P Water Temperature	24	SFP Water Sampling Analysis <still under debate >

1.4 Selection of Measuring Variables for BWR

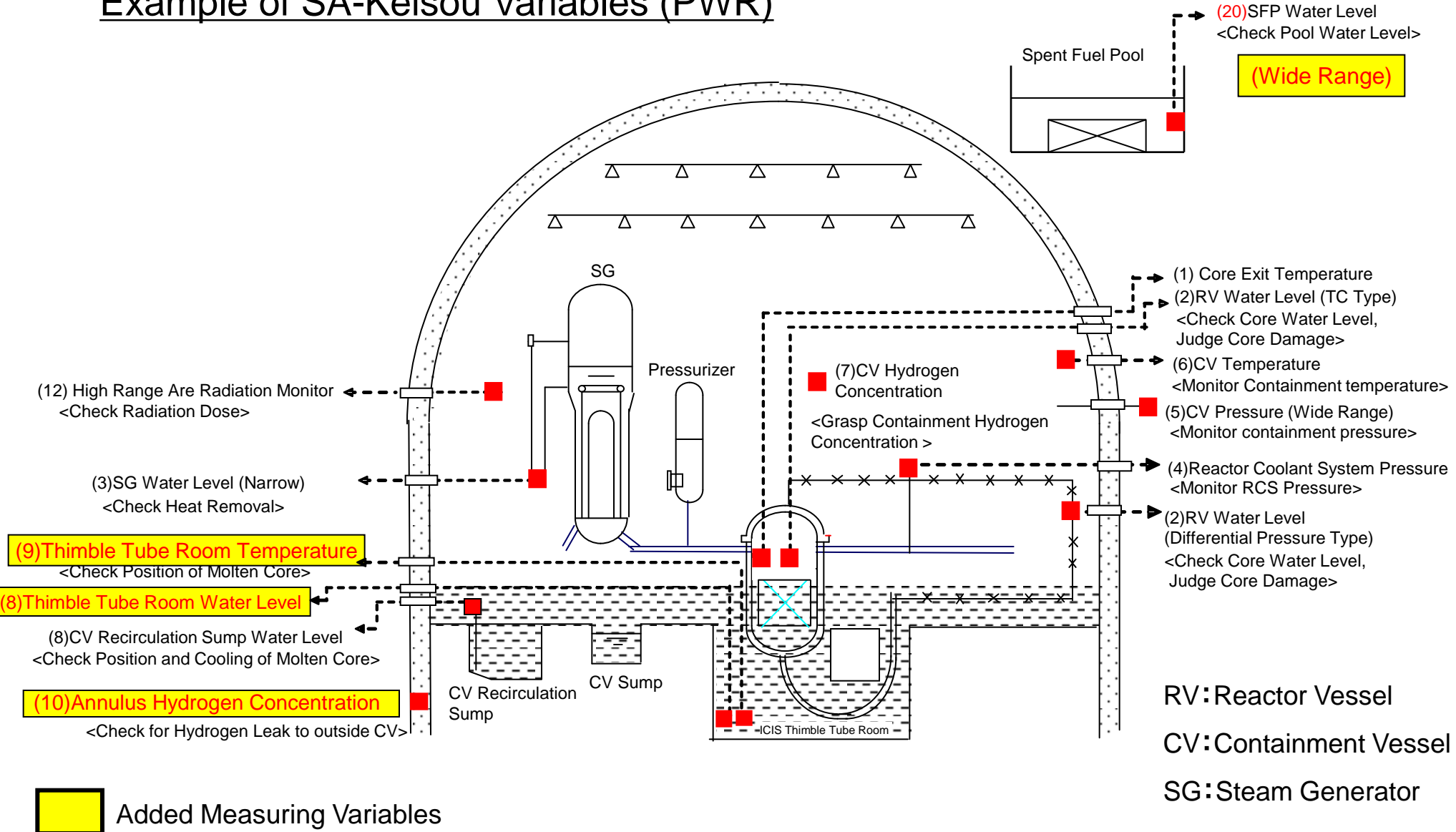
Example of SA-Keisou Variables (BWR)



● Selected Variables for PWR (Added Variables in red)

No	Variables	No	Variables
1	Core Exit Temperature	13	Exhaust High Range Gas Monitor
2	RV Water Level	14	Main Steam Line Monitor
3	SG Water Level (Narrow Range)	15	RWST Water Level
4	RCS Pressure	16	Neutron Monitor
5	CV Pressure (Wide Range)	17	CCW Surge Tank Pressure
6	CV Temperature	18	CCW Flow
7	CV Hydrogen Concentration	19	Fire Service Water Integrate Flow
8	ICIS Thimble Tube Room Water Level CV Recirculation Sump Water Level	20	SFP Water Level (Wide Range)
9	ICIS Thimble Tube Room Temperature	21	SFP Temperature
10	Annulus Hydrogen Concentration	22	SFP Area Monitor
11	Monitoring Post	23	SFP Hydrogen Concentration
12	High Range Area Radiation Monitor		

Example of SA-Keisou Variables (PWR)



1.5 SA Variables & SA Classification(1): Matrix of for BWR (Below is Draft Version)

Environment
Conditions

Measurement
Variables

(SA Classification) Plant condition	(SA1) SA1						(SA2) SA2						(SA3a) SA3a						(SA3b) SA3b									
	Core damage→Meltdown/RPV integrity is maintained/PCV integrity is maintained														Meltdown/RPV is damaged/PCV integrity is maintained						Meltdown/RPV is damaged/PCV is damaged						Meltdown/RPV is damaged/PCV is damaged	
Environmental condition in PCV •Maximum temperature •Pressure •Humidity •Radiation	•171 degree Celsius •0.31 MPa •Steam •5 × 106 Gy/6 months						•300 degree Celsius •1.0 MPa •Steam •5 × 106 Gy/6 months						•700 degree Celsius •1.0 MPa •Steam •5 × 106 Gy/6 months						•1000 degree Celsius •1.0 MPa •Steam •5 × 106 Gy/6 months									
Environmental condition outside PCV •Maximum temperature •Pressure •Humidity •Radiation	•66 degree Celsius •3.4 kPa •100% •3 × 105 Gy/6 months						•66 degree Celsius •0.01 MPa •Steam •3 × 105 Gy/6 months						•100 degree Celsius •0.01 MPa •Steam •2 × 106 Gy/6 months						•100 degree Celsius •0.01 MPa •Steam •2 × 106 Gy/6 months									
Required duration of performance	under debate																											
Measurement purpose / performance	Monitoring						Operation Confirmation						Monitoring						Operation Confirmation						Monitoring			
	Measurement Purposes / Performance	Possibility of boundary damage and confirmation of event initiation		Cooling function		Containment function		Operation Confirmation		Possibility of boundary damage and confirmation of event initiation		Cooling function		Containment function		Operation Confirmation		Possibility of boundary damage and confirmation of event initiation		Cooling function		Containment function		Operation Confirmation		Monitoring		
Confirm fuel boundary damage and meltdown		Confirm core cooling function	Confirm integrity of reactor coolant pressure boundary (RPV)	Confirm integrity of PCV	Confirm hydrogen inhibition within PCV	Injection to core	Depressurize RPV	PCV spray	PCV vent	Confirm damage of reactor coolant pressure boundary (RPV)	Confirm cooling of core debris outside RPV	Confirm integrity of PCV	Confirm hydrogen inhibition within PCV	Injection to core	Injection to pedestal	PCV spray	PCV vent	Confirm PCV damage	Confirm cooling of core debris outside RPV	Confirm emission of FP from PCV	Injection to core	Injection to pedestal	PCV spray	Control combustible gas concentration within R/B	Confirm cooling of core debris outside RPV			
No.																												
Parameters/Variables																												
1	RPV water level (RPV lower portion)	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙
2	RPV pressure	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙	⊙
3	RPV surface temperature	○	⊙	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○
4	D/W water level (AM) (Continuous)																											
5	D/W pressure (AM)																											
6	S/C waster level (AM)																											
7	S/C pressure (AM)																											
8	D/W temperature																											
9	D/W dose rate	⊙	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○
10	S/C dose rate	⊙	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○
11	D/W/S/C hydrogen concentration, oxygen concentration (GAMS)	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○	○
12	R/B area radiation monitor																											
13	Injection flow to RPV/ Flow of feedwater to RPV (AM)(Note 1)		○																									
14	Pedestal atmosphere temperature																											
15	Rod position indication value (Note 2)																											
16	CRDM indication value (Note 2)																											
17	Injection flow to pedestal/ Flow of feedwater to pedestal (AM)(Note 1)																											
18	S/P water temperature																											
19	S/C atmosphere temperature																											
20	R/B hydrogen concentration																											
21	RPV water sampling analysis/ Sampling of core water (Analysis of nuclear species, etc.)(Note 3)	○																										
	⊙: Main parameters required for accomplishing measurement purpose	◇: Available parameters for comprehending plant condition																										
	○: Dependent parameters required for accomplishing measurement purpose																											
	▲: Parameters required for initiating operation and confirming success of operation																											
	△: Parameters required for confirming success of operation																											
	Note 1: No.13 and 17 is preferable to be measured at feedwater line in front of RPV/PCV, but if feedwater line is established, outlet flow of resource of feedwater is supposed to be supplementally confirmed as a dependent parameter (backup parameter) and they are surveyed including that they are exluded from development object of phase I investigation.																											
	Note 2: No.15 and 16 are selected in existing AM (Accident Management) as parameters for confirming warning of RPV damage, but because they are for confirming numbers of loss of measurement and it is not necessary to enhance function of them, they are surveyed including that they are exluded from development object of phase I investigation.																											
	Note 3: No.21 is selected as reflection of Fukushima response, but because it is sampling analysis and is necessary after calming down of accident, it is surveyed including that they are exluded from development object of phase I investigation.																											
	Note 4: Resistance (critical value) is evaluated as instrumentation systems.																											

1.5 SA Variables & SA Classification(2): Matrix for PWR (Below is Draft Version)

Environment
Conditions

Measurement
Purposes /
Performance

Measurement Variables

(SA Classification) Plant condition		(SA1) SA1 Core damage→Meltdown/RV integrity is maintained/CV integrity is maintained				(SA2) SA2 Meltdown/RV is damaged/CV integrity is maintained				(SA3a) SA3a Meltdown/RV is damaged/CV is damaged				(SA3b) SA3b Meltdown/RV is damaged/CV is damaged									
Environmental condition in CV •Maximum temperature •Pressure •Humidity •Radiation		•190 degree Celsius •0.414 MPa •100% •Under existing PAM condition				below right SA3a condition				•200 degree Celsius •1.6 MPa •100% •2MGy/year (Annular space is under 5MGy/year.)				•300 degree Celsius •Atmosphere pressure •100% •2MGy/year (Annular space is under 5MGy/year.)									
Environment condition outside CV •Maximum temperature •Pressure •Humidity •Radiation		•Atmosphere temperature •Atmosphere pressure •-- •--				Same as right SA3a condition				•Depending on location •Atmosphere pressure •-- •Depending on location				•Depending on location •Atmosphere pressure •-- •Depending on location									
Required duration of performance		Monitoring				Monitoring				Monitoring				Monitoring									
Measurement purpose /performance		Possibility of boundary damage and confirmation of event initiation		Cooling function		Containment function		Operation Confirmation		Possibility of boundary damage and confirmation of event initiation		Cooling function		Containment function		Operation Confirmation		Possibility of boundary damage and confirmation of event initiation		Cooling function		Containment function	
		Confirm core damage		Confirm condition of core cooling		Confirm status of isolation function of containment		Confirm great deal of emission of FP from secondary system		Confirm loss of isolation function of containment		Confirm status of isolation function of containment		Confirm great deal of emission of FP from secondary system		Confirm loss of isolation function of containment		Confirm status of isolation function of containment		Confirm great deal of emission of FP from secondary system		Confirm loss of isolation function of containment	
No.	Parameters/Variables																						
1	Core exit temperature	⊙																					
2	Reactor vessel (RV) water level	⊙																					
3	Steam Generator (SG) water level (narrow range)		⊙																				
4	Reactor coolant pressure/RCS pressure		⊙																				
5	Containment pressure/CV pressure (wide range)			⊙																			
6	Containment temperature/CV temperature																						
7	Containment hydrogen concentration/ CV hydrogen concentration																						
8-1	Reactor cavity water level/ ICIS Thimble Tube Room water level (Note 2)																						
8-2	Containment recirculation sump water level / CV recirculation sump water level (Note 2)																						
9	Reactor cavity temperature/ ICIS Thimble Tube Room temperature																						
10	Annulus hydrogen concentration																						
11	Monitoring post (Note 3)																						
12	Containment high range area radiation monitor/ High range area radiation monitor		⊙																				
13	Containment exhaust radiation gas monitor/ Exhaust high range gas monitor																						
14	Main steam line radiation monitor																						
15	RWST water level																						
16	NIS																						
17	CCW surge tank pressure (wide range)																						
18	CCW flow																						
19	Fire service water accumulated flow/Integrate flow																						

- ⊙ : Main parameters required for accomplishing measurement purpose
- : Dependent parameters required for accomplishing measurement purpose
- ▲ : Parameters required for initiating operation and confirming success of operation
- △ : Parameters required for confirming success of operation
- ◇ : Available parameters for comprehending plant condition

Environment resistance is required in accordance with survey result of 2011.
Environment resistance is required with limiting duration required for SA operation.

Note 1 Confirm containment pressure instead of reactor coolant pressure after RV damage (because reactor coolant pressure quickly becomes same pressure as containment pressure after RV damage.)
Therefore, environment resistance of reactor coolant is included to verification condition of PAM.

Note 2 Reactor cavity water level overlaps existing measurement range of recirculation sump water level. Existing recirculation sump water level measurement utilizes diaphragm seal sensor and because it is thought that it is likely to be influenced by variation of environmental condition, reactor cavity water level is measured by applying such as thermocouple type.

Note 3 Monitoring post is surveyed including excluding from development object because it is located outside CV and it is possible to apply equipment such as portable type.
In addition, it is not limited to monitoring post if it can measure dose rate outside CV.

Operation Confirmation
↓

Same as Note 4

1.6 Design Requirements of Industrial Proposal(1)

NO.	Items	Requirements
1	Independence between Levels of Defense in Depth	Level of defense in depth of main parameters, which are categorized as Level 4, should be designed to have independence from Level 3 parameters. The definition of Level 3 and 4 is consistent with the defense in depth level identified in IAEA SSR-2/1.
2	Redundancy and Diversity	Redundancy and diversity requirements same as safety-related equipment are not required. Basically single configuration is applicable. However, the severe accident monitoring instrumentation should be designed to meet enough reliability for monitoring.
3	Separation and Independence	Level 4 parameters should be designed to have independence from level 3 parameters wherever practical. Redundant equipment should be independent and separated from each other wherever practical.
4	Environmental Qualification	Severe accident monitoring instrumentation should be designed to have environmental resistance, including temperature, pressure, and radiation. Conditions of temperature, pressure, and radiation under severe accident environment should be set with considering SA classification and arrangement location.
5	Seismic Qualification	Seismic qualification should be required.

1.6 Design Requirements of Industrial Proposal(2)

NO.	Items	Requirements
6	Power Supply	The power supply for severe accident monitoring instrumentation should be used.
7	Testability	Severe accident monitoring instrumentation should be designed to have periodic testing capability when the reactor is in operation or in shutdown.
8	Quality Assurance	Quality assurance such as design control should be conducted in accordance with JEAG 4111-2009 (Guideline for quality assurance widely used for Japanese nuclear power plants).
9	Indication and Record	Severe accident monitoring instrumentation should be designed to provide continuous monitoring display in the main control room and recording capability should be provided.
10	Flooding Protection	If internal flooding and external flooding occur, required flooding protection should be taken to accomplish function of severe accident monitoring instrumentation.
11	Fire Protection	Severe accident monitoring instrumentation should be designed that noncombustible and fire retardant materials are used wherever practical.

- The development of the SA-Keisou is needed to monitor the important parameters which contribute to the prevention of severe accidents at the plant.
- Extraction of measuring variables for SA-Keisou including additional variables is accomplished based on analysis of accident management guidance (AMG), and major standards, the analysis of TEPCO's Fukushima Daiichi accident.
- Measuring variables for SA-Keisou have been determined.
 - BWR : 24 variables (4 added variables)
 - PWR : 23 variables (4 added variables)
- In the severe accident instrumentation, plant conditions after core damage are classified into the following:
 - SA1: Core is damaged, but core fuel is inside the reactor vessel
 - SA2: RV failure; core fuel is outside the reactor vessel
 - SA3a: Containment failure (Water Feed Injection to inside of CV is success)
 - SA3b: Containment failure (Water Feed Injection to inside of CV is failure)
- The parameters in relation to SA classification depending on plant situation, including measurement purpose such as monitoring and operation confirmation were also shown.

R&D Plan contents	Fiscal 2011	Fiscal 2012	Fiscal 2013	Fiscal 2014
1. Defining the requirements for instrumentation systems				
① Selection of parameters required for SA	■			
② Setting for the environment conditions and etc of SA	■			
③ Defining the requirements		■		
2. The settling of the primary development plan				
① Public Offering of Technical Ideas		■		
② Setting for the basic specification on instrumentation systems		■		
③ Study of the development plan on instrumentation systems		■		
④ Investigation of the priority of the development plan		■		
3. The development of the SA instrumentation systems				
① Design and prototype production of instrumentation systems		■		
② Basic test and analysis on instrumentation systems		■		
③ Qualification tests on instrumentation systems			■	
4. Drawing up standards and guidelines for SA				
① Investigation of overseas standards		■		
② Drawing up standards and guidelines		■		

Phase 1: 2011 to 2nd quarter 2014

System Installation for actual NPPs : after completing tests (from 2013)

Thank you for your attention

END

終 (O-wa-ri)